

## SELECTION OF IMPORTANT MONTE CARLO HISTORIES

Stephen D. Egbert

*Science Applications International Corporation*

---

The 1986 Dosimetry System (DS86) for Japanese A-bomb survivors uses information describing the behavior of individual radiation particles, simulated by Monte Carlo methods, to calculate the transmission of radiation into structures and, thence, into humans. However, there are practical constraints on the number of such particle "histories" that may be used. First, the number must be sufficiently high to provide adequate statistical precision for any calculated quantity of interest. For integral quantities, such as dose or kerma, statistical precision of approximately 5% (standard deviation) is required to ensure that statistical uncertainties are not a major contributor to the overall uncertainty of the transmitted value. For differential quantities, such as scalar fluence spectra, 10 to 15% standard deviation on individual energy groups is adequate. Second, the number of histories cannot be so large as to require an unacceptably large amount of computer time to process the entire survivor data base. Given that there are approximately 30,000 survivors, each having 13 or 14 organs of interest, the number of histories per organ must be constrained to less than several ten's of thousands at the very most.

Selection and use of only the most important Monte Carlo leakage histories from among all those calculated allows the creation of an efficient house and organ radiation transmission system for use at RERF. While attempts have been made during the adjoint Monte Carlo calculation to bias the histories toward an efficient dose estimate, this effort has been far from satisfactory. Many of the adjoint histories on a typical leakage tape are either starting in an energy group in which there is very little kerma or dose or leaking into an energy group with very little free-field fluence to couple with. By knowing the typical free-field fluence and the fluence-to-dose factors with which the leaking histories will be used, one can select histories from a leakage tape that will contribute to dose and reject the histories which add little dose information. As will be shown, a large number of histories can be rejected without causing an unacceptable increase in the statistical uncertainty of the desired dose calculation.

### Method

The history selection method can be outlined as follows:

1. Run an adjoint Monte Carlo transport code to produce a leakage tape that has more than enough histories to sift through (e.g., 400,000 to 600,000).
2. Use a forward-adjoint fluence coupling code to couple this leakage tape to a free-field fluence tape. Score the dose according to each starting and leaking energy group combination. Also calculate the variance of each of these doses by analyzing the variation of the dose between the batches of histories.
3. Minimize the Monte Carlo uncertainty on the total dose by selecting a subset of histories such that the number of histories for each combination of starting-leaking energy groups are proportional to  $DOSE_{ij} * FSD_{ij} * \sqrt{N_{ij}}$ .

The  $DOSE_{ij}$  is the dose resulting from free-field fluence group  $i$  resulting in dose in kerma group  $j$ .  $FSD_{ij}$  is the Monte Carlo fractional standard deviation of  $DOSE_{ij}$ . An  $N_{ij}$  is the number of histories needed to get  $FSD_{ij}$ .

### Mathematical Basis

The mathematical basis for this procedure is as follows. The total dose from a forward-adjoint coupling calculation is determined by:

$$\begin{aligned} \text{Total dose} &= \sum_j R_j \phi_j \\ &= \sum_j R_j \sum_i \phi_i T_{ij} \end{aligned} \quad (1)$$

where

- $\phi_i$  = free field fluence for energy group  $i$ ,
- $\phi_j$  = shielded fluence for energy group  $j$ ,
- $T_{ij}$  = fluence transfer from group  $i$  (free field) to group  $j$  (shielded), and
- $R_j$  = response function for each energy group  $j$ .

If it is assumed that the variance of a transfer matrix element  $T_{ij}$  is independent of other elements, then the total variance of the dose can be calculated by:

$$\begin{aligned} \sigma^2 &= \sum_j R_j^2 \sigma_j^2 \\ &= \sum_j R_j^2 \sum_i (\phi_i \sigma_{ij})^2 \\ &= \sum_j R_j^2 \sum_i (\phi_i T_{ij} FSC_{ij})^2 \end{aligned} \quad (2)$$

where

- $\sigma^2$  = the variance of the total dose,
- $\sigma_j^2$  = the variance of shielded fluence group  $j$ ,
- $\sigma_{ij}^2$  = the variance of the individual transfer matrix element  $T_{ij}$ , and
- $FSD_{ij}$  = the FSD of transfer matrix element  $T_{ij}$   
(it is the common output statistic of the DRC coupling system).

Finally, it is assumed that the  $FSD_{ij}$  is proportional, with constants of proportionality  $K_{ij}$ , to the inverse square root of the number of histories  $N_{ij}$  on the adjoint leakage tape going from group  $i$  to group  $j$ ,

$$FSD_{ij} = \frac{K_{ij}}{\sqrt{N_{ij}}} \quad (3)$$

While the assumption of independence may not be entirely accurate, the results of our assumption can be tested later with real calculations.

Minimization of  $\sigma^2$  with respect to the number of histories going from  $k$  to  $l$ ,  $N_{kl}$ , will provide an optimum number of histories in each element of the transfer matrix. This is accomplished as follows:

$$\begin{aligned} \frac{\delta \sigma^2}{\delta N_{kl}} &= 0 \\ &= \sum_j R_j^2 \frac{\delta \sigma_j^2}{\delta N_{kl}} = \sum_j R_j^2 \sum_i \phi_i^2 T_{ij}^2 \frac{\delta FSD_{ij}^2}{\delta N_{kl}} \end{aligned}$$

From Equation (3)

$$0 = \sum_j R_j^2 \sum_i \phi_i^2 T_{ij}^2 K_{ij}^2 \frac{\sigma \left[ \frac{1}{N_{ij}} \right]}{N_{kl}} = - \sum_j R_j^2 \sum_i \frac{\sigma_i^2 T_{ij}^2 K_{ij}^2}{N_{ij}^2} \frac{\sigma N_{ij}}{\sigma N_{kl}} \quad (4)$$

Now evaluate:  $\delta N_{ij} / \delta N_{kl}$ .

$$\frac{\delta N_{ij}}{\delta N_{kl}} = 1, \text{ if } i = k \text{ and } j = l,$$

but

$$\frac{\delta N_{ij}}{\delta N_{kl}} = -C_{ijkl}, \text{ if } i \neq k \text{ or } j \neq l.$$

Also

$$N = \sum_j \sum_i N_{ij} \quad (5)$$

$$\frac{\delta N}{\delta N_{kl}} = 0 = \sum_j \sum_i \frac{\delta N_{ij}}{\delta N_{kl}} = \left[ \sum_{j \neq i} \sum_{i \neq k} \frac{\delta N_{ij}}{\delta N_{kl}} \right] + \frac{\delta N_{ij}}{\delta N_{kl}}$$

$$0 = \left[ \sum_{j \neq 1} \sum_{i \neq k} -C_{ijkl} \right] + 1 \quad (6)$$

Thus, on returning to the minimization of  $\sigma^2$ ,

$$\frac{R_1^2 \phi_k^2 T_{kl}^2 K_{kl}^2}{N_{kl}^2} = \sum_{j \neq 1} R_j^2 \sum_{i \neq 1} \frac{\delta_i^2 T_{ij}^2 K_{ij}^2 C_{ijkl}^2}{N_{ij}^2}$$

This is solved when

$$N_{kl}^2 = R_1^2 \phi_k^2 T_{kl}^2 K_{kl}^2 A \quad (7)$$

where A is a proportionality constant that can be used for normalizing the total number of histories. To determine the constant  $K_{kl}$ , a coupling calculation can be used to which calculates  $FSD_{kl}^o$  from a number of histories  $N_{kl}^o$ . Then:

$$K_{kl}^2 = FSD_{kl}^{o2} N_{kl}^o \quad (8)$$

$$N_{kl}^2 = R_1^2 \phi_k^2 T_{kl}^2 FSD_{kl}^{o2} N_{kl}^o A \quad (9)$$

A is determined from Equation (5)

$$N = A \sum_1 R_1 \sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o} \quad (10)$$

and

$$A = \frac{N}{\sum_1 R_1 \sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (11)$$

Thus, the optimal number  $N_{kl}$  is found to be:

$$N_{kl} = N \frac{R_1 \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}}{\sum_1 R_1 \sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (12)$$

This expression can be evaluated in a two-step process to separate the process of free-field fluence optimization from that of dose-response optimization. Rewriting the expression in two parts is done as follows:

$$N_{kl} = N_1 \frac{\phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}}{\sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (13)$$

$$N_1 = N \frac{R_1 \sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}}{\sum_1 R_1 \sum_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (14)$$

The first expression selects the number of histories leaking in each free-field group k given a shielded group l. The second expression tells how to optimize the various shielded groups. The operation could be performed also in one step if desired.

The two equations above have also been modified so that emphasis can be put on part of the free-field spectrum or on part of the response function.



$$N_{kl} = N_1 \frac{F_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}}{\sum_k F_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (15)$$

$$N_1 = N \frac{M_1 R_1 \sum_k F_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}}{\sum_l M_1 R_1 \sum_k F_k \phi_k T_{kl} FSD_{kl}^o \sqrt{N_{kl}^o}} \quad (16)$$

were  $F_k$  is a fluence multiplier for group  $k$  and  $M_1$  is a response multiplier for group  $l$ . This modification is necessary so that in situations where certain parts of the spectrum (e.g., neutrons) do not contribute much dose, their contribution to improve their statistics can be arbitrarily increased.

Another practical concern is to ensure that  $N_{kl}$ , the number of desired histories, is not larger than  $N_{kl}^o$ , the number of available histories on the large leakage tape. Also, it might be wise not to reject too many of the unimportant histories because as the number of histories in any transfer matrix element is reduced the chance of a Monte Carlo anomaly increases. Then the hoped for improvement in statistics may fail to appear.

### Example

The optimization procedure was carried out for a phantom organ adjoint data set placed in the Hiroshima free field. The result illustrates the use and benefit of this method for reducing coupling time.

First, an adjoint MORSE-VCS calculation was performed to produce a large-number-of-histories leakage tape. The specific details are as follows:

Phantom: Japanese adult  
 Organ: marrow  
 Starting histories: 400,000  
 Leaking histories: 582,233  
 Orientation: left side to burst, standing.

Second, a tape of free-field fluence was prepared. The specific details are as follows:

City: Hiroshima  
 Ground range: 1500 m  
 Radiation: neutrons, prompt and secondary gamma rays.

The phantom shielding and free-field fluences were coupled using the DRC code that is part of the VCS system. Modifications were made to calculate also the fluence transfer matrix,  $T_{kl}$ ; the number matrix,  $N_{kl}^o$ ; and the FSD matrix,  $FSD_{kl}^o$ , in addition to the usual DRC output. Table 1 shows the main radiation doses and the FSD associated with them. Tables 2 to 4 show the  $T_{kl}$ ,  $N_{kl}^o$ , and  $FSD_{kl}^o$  obtained from this coupling that used all the histories.

Based on the experience at RERF this coupling would cost approximately two minutes of CPU time and eight minutes of clock time per individual organ. For 10 organs and 10,000 survivors that would be about 5 months of CPU time and 18 months of clock time. Such long times are unacceptable for practice.

Table 1. Comparison of Organ Doses and FSDs with Various History Biasing Schemes. Hiroshima, 1500m Ground Range, Marrow ( $\approx 600,000$  Leaking Histories)

	99% Reduction in Histories						Dec 84 System
	All Histories	Random Rejection	Leaking Bias	Starting Bias Also	Leaking* Bias	Starting* Bias Also	
DOSES (rad)							
Neutron	.261	.241	.265	.269	.259	.259	.274
Prompt Gamma	19.104	20.460	16.762	18.553	18.559	18.918	18.859
Debris Gamma	23.681	24.495	23.934	21.844	21.526	22.864	25.101
Body Gamma	.198	.189	.262	.199	.180	.205	.157
FSDs (in percent)							
Neutron	1.0(.9)	10.4(9.4)	5.9(5.9)	24.3(27.5)	5.7(5.9)	4.8(4.9)	4.9
Prompt Gamma	1.4(1.5)	13.3(14.8)	9.2(11.1)	4.4(4.6)	11.6(11.3)	5.3(5.7)	5.8
Debris Gamma	1.5	13.9	8.7	6.01	10.1	7.7	6.1
Body Gamma	2.1(1.2)	9.8(11.9)	21.4(29.9)	31.1(30.2)	7.5(7.1)	5.3(5.3)	8.1
# of Histories	582233	5822	5822	5822	5822	5822	29549
Estimated Clock Time for 10 organs 10,000 survivors	18 month	6 days	6 days	6 days	6 days	6 days	30 days

\*Neutron fluence multiplied by 50.0 to improve statistics on neutron components.

( ) values in parenthesis indicate estimated values by summing individual variances.

A prediction of the total FSDs on the major dose components was made by summing the variances in Tables 2 to 4. This procedure assumes a reasonable accuracy in the FSDs. Table 1 indicates the predicted FSD results in parentheses. The comparison with actual FSDs is good except for body gamma. It is likely that the exception is caused by the large FSDs in the transfer matrix of this dose component.

Next, the results of several history biasing methods were illustrated for a 99% reduction in the number of histories. This smaller number of histories reduces the estimated clock time to about four seconds per individual per organ. It would reduce the total time to do 10 organs for 10,000 survivors to six days.

Random rejection of histories to leave only 5,822 of them results in doses and FSDs as shown in Table 1. The new matrices  $N_{kl}$  and  $FSD_{kl}$  are shown in Tables 5 and 6.

Next, histories were selected based on the importance of their leaking energy. The doses and FSDs are shown in Table 1. The new matrices for  $N_{kl}$  and  $FSD_{kl}$  are shown in Tables 7 and 8. Note the different emphasis on the number of histories leaking out at large neutron energies. Because there is little free-field fluence at the high energies, very few histories are selected that leak at high energies.

Finally, histories were chosen based on the importance of their leaking energy and on their starting energy. The major doses and FSDs are again shown in Table 1. The new matrices  $N_{kl}$  and  $FSD_{kl}$  are shown in Tables 9 and 10. Note that this time there are very few neutrons histories at all. This is because the neutron dose is only a small part of the total dose. The loss of histories is unacceptable because, while the neutrons are relatively unimportant in the total kerma or dose, they are important because they produce more effect per unit dose. Thus, the histories were reselected based on the importance of their leaking











Table 4. FSD Transfer Matrix,  $FSD_{kl}$ , for Hiroshima, 1500 m Ground Range, Coupled with Marrow Leakage Tape (582,223 Histories)

LEAKING ENERGY GROUP	10										20										30										40										50										50									
	1	2	3	4	5	6	7	8	9	10	1	2	3	4	5	6	7	8	9	10	1	2	3	4	5	6	7	8	9	10	1	2	3	4	5	6	7	8	9	10	1	2	3	4	5	6	7	8	9	10	1	2	3	4	5	6	7	8	9	10
1	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000

459

Table 5. Number Transfer Matrix,  $N_{kl}$ , for Hiroshima, 1500 m Ground Range. Fluence Coupled with Randomly Reduced Marrow Leakage Tape (5,822 Histories)

UNIFORMLY REDUCED NUMBER OF HISTORIES	LEAKING ENERGY GROUP									
	1	2	3	4	5	6	7	8	9	10
1	1	0	0	0	0	0	0	0	0	0
2	0	1	0	0	0	0	0	0	0	0
3	0	0	1	0	0	0	0	0	0	0
4	0	0	0	1	0	0	0	0	0	0
5	0	0	0	0	1	0	0	0	0	0
6	0	0	0	0	0	1	0	0	0	0
7	0	0	0	0	0	0	1	0	0	0
8	0	0	0	0	0	0	0	1	0	0
9	0	0	0	0	0	0	0	0	1	0
10	0	0	0	0	0	0	0	0	0	1
11	0	0	0	0	0	0	0	0	0	0
12	0	0	0	0	0	0	0	0	0	0
13	0	0	0	0	0	0	0	0	0	0
14	0	0	0	0	0	0	0	0	0	0
15	0	0	0	0	0	0	0	0	0	0
16	0	0	0	0	0	0	0	0	0	0
17	0	0	0	0	0	0	0	0	0	0
18	0	0	0	0	0	0	0	0	0	0
19	0	0	0	0	0	0	0	0	0	0
20	0	0	0	0	0	0	0	0	0	0
21	0	0	0	0	0	0	0	0	0	0
22	0	0	0	0	0	0	0	0	0	0
23	0	0	0	0	0	0	0	0	0	0
24	0	0	0	0	0	0	0	0	0	0
25	0	0	0	0	0	0	0	0	0	0
26	0	0	0	0	0	0	0	0	0	0
27	0	0	0	0	0	0	0	0	0	0
28	0	0	0	0	0	0	0	0	0	0
29	0	0	0	0	0	0	0	0	0	0
30	0	0	0	0	0	0	0	0	0	0
31	0	0	0	0	0	0	0	0	0	0
32	0	0	0	0	0	0	0	0	0	0
33	0	0	0	0	0	0	0	0	0	0
34	0	0	0	0	0	0	0	0	0	0
35	0	0	0	0	0	0	0	0	0	0
36	0	0	0	0	0	0	0	0	0	0
37	0	0	0	0	0	0	0	0	0	0
38	0	0	0	0	0	0	0	0	0	0
39	0	0	0	0	0	0	0	0	0	0
40	0	0	0	0	0	0	0	0	0	0
41	0	0	0	0	0	0	0	0	0	0
42	0	0	0	0	0	0	0	0	0	0
43	0	0	0	0	0	0	0	0	0	0
44	0	0	0	0	0	0	0	0	0	0
45	0	0	0	0	0	0	0	0	0	0
46	0	0	0	0	0	0	0	0	0	0
47	0	0	0	0	0	0	0	0	0	0
48	0	0	0	0	0	0	0	0	0	0
49	0	0	0	0	0	0	0	0	0	0
50	0	0	0	0	0	0	0	0	0	0

UNIFORM NUMBER OF HISTORIES











Table 9. Number Transfer Matrix,  $N_{kl}$ , for Hiroshima, 1500 m Ground Range. Fluence Coupled with Optimized Leaking and Starting History Marrow Leakage Tape (5,822 Histories)

STARTING HISTORIES NUMBER OF HISTORIES	OPTIMIZED RESULTS DISPLAYED	LEAKING ENERGY GROUP	20	30	40	50	60	70	80	90	100
1	1	1	1	1	1	1	1	1	1	1	1
2	2	2	2	2	2	2	2	2	2	2	2
3	3	3	3	3	3	3	3	3	3	3	3
4	4	4	4	4	4	4	4	4	4	4	4
5	5	5	5	5	5	5	5	5	5	5	5
6	6	6	6	6	6	6	6	6	6	6	6
7	7	7	7	7	7	7	7	7	7	7	7
8	8	8	8	8	8	8	8	8	8	8	8
9	9	9	9	9	9	9	9	9	9	9	9
10	10	10	10	10	10	10	10	10	10	10	10
11	11	11	11	11	11	11	11	11	11	11	11
12	12	12	12	12	12	12	12	12	12	12	12
13	13	13	13	13	13	13	13	13	13	13	13
14	14	14	14	14	14	14	14	14	14	14	14
15	15	15	15	15	15	15	15	15	15	15	15
16	16	16	16	16	16	16	16	16	16	16	16
17	17	17	17	17	17	17	17	17	17	17	17
18	18	18	18	18	18	18	18	18	18	18	18
19	19	19	19	19	19	19	19	19	19	19	19
20	20	20	20	20	20	20	20	20	20	20	20
21	21	21	21	21	21	21	21	21	21	21	21
22	22	22	22	22	22	22	22	22	22	22	22
23	23	23	23	23	23	23	23	23	23	23	23
24	24	24	24	24	24	24	24	24	24	24	24
25	25	25	25	25	25	25	25	25	25	25	25
26	26	26	26	26	26	26	26	26	26	26	26
27	27	27	27	27	27	27	27	27	27	27	27
28	28	28	28	28	28	28	28	28	28	28	28
29	29	29	29	29	29	29	29	29	29	29	29
30	30	30	30	30	30	30	30	30	30	30	30
31	31	31	31	31	31	31	31	31	31	31	31
32	32	32	32	32	32	32	32	32	32	32	32
33	33	33	33	33	33	33	33	33	33	33	33
34	34	34	34	34	34	34	34	34	34	34	34
35	35	35	35	35	35	35	35	35	35	35	35
36	36	36	36	36	36	36	36	36	36	36	36
37	37	37	37	37	37	37	37	37	37	37	37
38	38	38	38	38	38	38	38	38	38	38	38
39	39	39	39	39	39	39	39	39	39	39	39
40	40	40	40	40	40	40	40	40	40	40	40
41	41	41	41	41	41	41	41	41	41	41	41
42	42	42	42	42	42	42	42	42	42	42	42
43	43	43	43	43	43	43	43	43	43	43	43
44	44	44	44	44	44	44	44	44	44	44	44
45	45	45	45	45	45	45	45	45	45	45	45
46	46	46	46	46	46	46	46	46	46	46	46
47	47	47	47	47	47	47	47	47	47	47	47
48	48	48	48	48	48	48	48	48	48	48	48
49	49	49	49	49	49	49	49	49	49	49	49
50	50	50	50	50	50	50	50	50	50	50	50
51	51	51	51	51	51	51	51	51	51	51	51
52	52	52	52	52	52	52	52	52	52	52	52
53	53	53	53	53	53	53	53	53	53	53	53
54	54	54	54	54	54	54	54	54	54	54	54
55	55	55	55	55	55	55	55	55	55	55	55
56	56	56	56	56	56	56	56	56	56	56	56
57	57	57	57	57	57	57	57	57	57	57	57
58	58	58	58	58	58	58	58	58	58	58	58
59	59	59	59	59	59	59	59	59	59	59	59
60	60	60	60	60	60	60	60	60	60	60	60
61	61	61	61	61	61	61	61	61	61	61	61
62	62	62	62	62	62	62	62	62	62	62	62
63	63	63	63	63	63	63	63	63	63	63	63
64	64	64	64	64	64	64	64	64	64	64	64
65	65	65	65	65	65	65	65	65	65	65	65
66	66	66	66	66	66	66	66	66	66	66	66
67	67	67	67	67	67	67	67	67	67	67	67
68	68	68	68	68	68	68	68	68	68	68	68
69	69	69	69	69	69	69	69	69	69	69	69
70	70	70	70	70	70	70	70	70	70	70	70
71	71	71	71	71	71	71	71	71	71	71	71
72	72	72	72	72	72	72	72	72	72	72	72
73	73	73	73	73	73	73	73	73	73	73	73
74	74	74	74	74	74	74	74	74	74	74	74
75	75	75	75	75	75	75	75	75	75	75	75
76	76	76	76	76	76	76	76	76	76	76	76
77	77	77	77	77	77	77	77	77	77	77	77
78	78	78	78	78	78	78	78	78	78	78	78
79	79	79	79	79	79	79	79	79	79	79	79
80	80	80	80	80	80	80	80	80	80	80	80
81	81	81	81	81	81	81	81	81	81	81	81
82	82	82	82	82	82	82	82	82	82	82	82
83	83	83	83	83	83	83	83	83	83	83	83
84	84	84	84	84	84	84	84	84	84	84	84
85	85	85	85	85	85	85	85	85	85	85	85
86	86	86	86	86	86	86	86	86	86	86	86
87	87	87	87	87	87	87	87	87	87	87	87
88	88	88	88	88	88	88	88	88	88	88	88
89	89	89	89	89	89	89	89	89	89	89	89
90	90	90	90	90	90	90	90	90	90	90	90
91	91	91	91	91	91	91	91	91	91	91	91
92	92	92	92	92	92	92	92	92	92	92	92
93	93	93	93	93	93	93	93	93	93	93	93
94	94	94	94	94	94	94	94	94	94	94	94
95	95	95	95	95	95	95	95	95	95	95	95
96	96	96	96	96	96	96	96	96	96	96	96
97	97	97	97	97	97	97	97	97	97	97	97
98	98	98	98	98	98	98	98	98	98	98	98
99	99	99	99	99	99	99	99	99	99	99	99
100	100	100	100	100	100	100	100	100	100	100	100

GROUP NUMBER DISPLAYED



Table 11. Number Transfer Matrix,  $N_{kl}$ , for Hiroshima, 1500 m Ground Range. Fluence (Neutrons Increased 50 Times) Coupled with Optimized Leaking History Marrow Leakage Tape (5,822 Histories)

LEAKING HISTORIES OPTIMIZED RESULTS DISPLAYED NUMBER OF HISTORIES	LEAKING ENERGY GROUP		20	50	40	50	58
	20	50					
1	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
2	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
3	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
4	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
5	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
6	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
7	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
8	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
9	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
10	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
11	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
12	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
13	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
14	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
15	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
16	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
17	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
18	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
19	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
20	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
21	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
22	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
23	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
24	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
25	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
26	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
27	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
28	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
29	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
30	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
31	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
32	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
33	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
34	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
35	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
36	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
37	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
38	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
39	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
40	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
41	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
42	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
43	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
44	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
45	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
46	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
47	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
48	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
49	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
50	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
51	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
52	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
53	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
54	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
55	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
56	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
57	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
58	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000

LEAKING ENERGY GROUP







Table 14. FSD Transfer Matrix, FSD<sub>K1</sub>, for Hiroshima, 1500 m Ground Range. Fluence (Neutrons Increased 50 Times) Coupled with Optimized Leaking and Starting History Marrow Leakage Tape (5,822 Histories)

Table with columns for LEAKING ENERGY GROUP (1-58) and rows for various energy groups (1-58). The table contains numerical data representing the transfer matrix elements.

LEAKING ENERGY GROUP



energy with the neutrons increased arbitrarily by 50 times. Table 1 shows the major doses and FSDs. Tables 11 and 12 show the matrices  $N_{kl}$  and  $FSD_{kl}$ .

Finally, histories were chosen based on both the importance of their leaking and starting energies, still with the neutron importance increased by 50. Table 1 shows the major doses and FSDs. Tables 13 and 14 show the matrices  $N_{kl}$  and  $FSD_{kl}$ . This is the method of biasing that we recommend for calculating the dose in the phantom.

The final column of Table 1 gives the results from the RERF coupling system used in December 1984, which used unbiased leakage data sets of approximately 30,000 histories each. The doses are about the same in spite of minor phantom and organ distribution changes. The FSDs are comparable to those for the recommended biasing method and the clock time for the recommended biasing method is about one fifth that experienced using the unbiased system.

### Concluding Comments

By preprocessing the leakage tape the important histories can be saved and used in calculating dose. Some information is sacrificed of course. Energy groups with small doses will have very large FSDs. The tapes optimized for calculating dose are thus unusable for calculating spectra. If spectra are desired a similar optimization procedure for dose in each energy group is available. For example, roughly 40,000 histories are required to get a 10% FSD in each of the DLC-31 groups.

As the number of histories used is reduced, the Monte Carlo FSDs increase. Figures 1 and 2 show the result for reducing the number of histories from a typical organ calculation. In Figure 1 the FSDs for each dose component are plotted as a function of the number of histories making up that dose component. The number of neutron and neutron-induced gamma-ray histories can be reduced by a factor of 10 or the gamma-ray histories reduced by a factor of 5 before dose-important histories are removed. Thereafter the FSD will be inversely proportional to the square root of the number of histories and will, therefore, increase as the number of histories is reduced. In the system, the neutron and gamma-ray histories compete for importance. Figure 2 shows how the dose FSD is changed as the total number of all histories is reduced. (The neutron's dose importance has been set at 20 times that of the gamma rays.) A reduction to 1/3 of the number of histories makes no difference in the FSD. A further reduction of 1/20 realigns the dose components according to their importance to dose. Any further reduction just causes the same increase in FSDs as stated above.

For the purposes of creating data sets for DS86, history biasing was used based on the Hiroshima spectrum at 1500 m ground range. To get adequate neutron statistics the neutron fluence was increased by a factor of 20 for the house data sets and 20 for the organ tape. The house histories were biased only for the leaking energies with the free field because this provided a respectable spectrum in every energy group. For each organ, two data sets were produced. One was biased for dose only. The other was biased for spectra in all energy groups. The house data sets consist of about 40,000 histories each. The organ data sets consist of about 6,000 histories per organ for dose and 40,000 histories per organ for spectra. The user may select the data appropriate to his needs. It must be pointed out that the doses calculated from the two organ tapes will be similar but not exactly the same because of the different histories used on the two tapes.

SELECTION OF MONTE CARLO HISTORIES

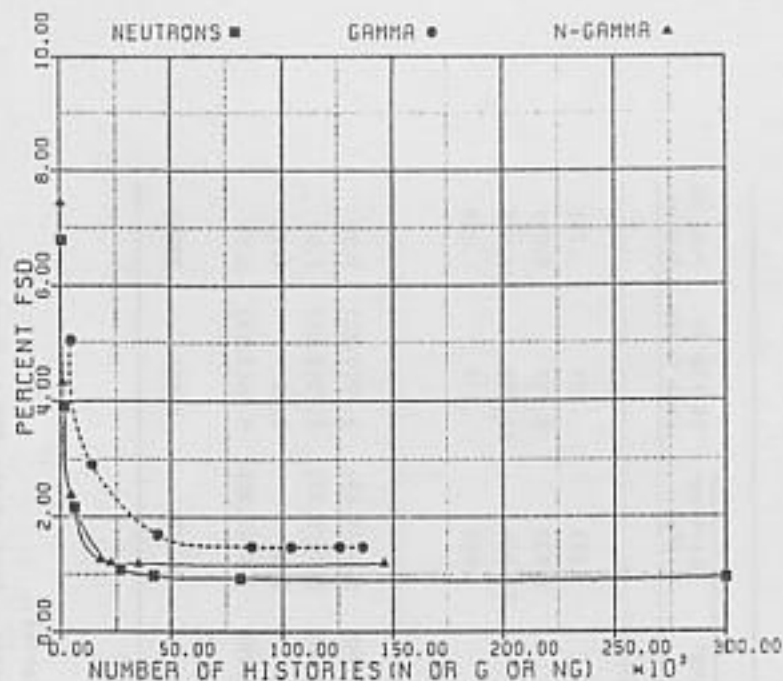


Figure 1. Percent fractional standard deviation as a function of history reduction. (N or G or NG)

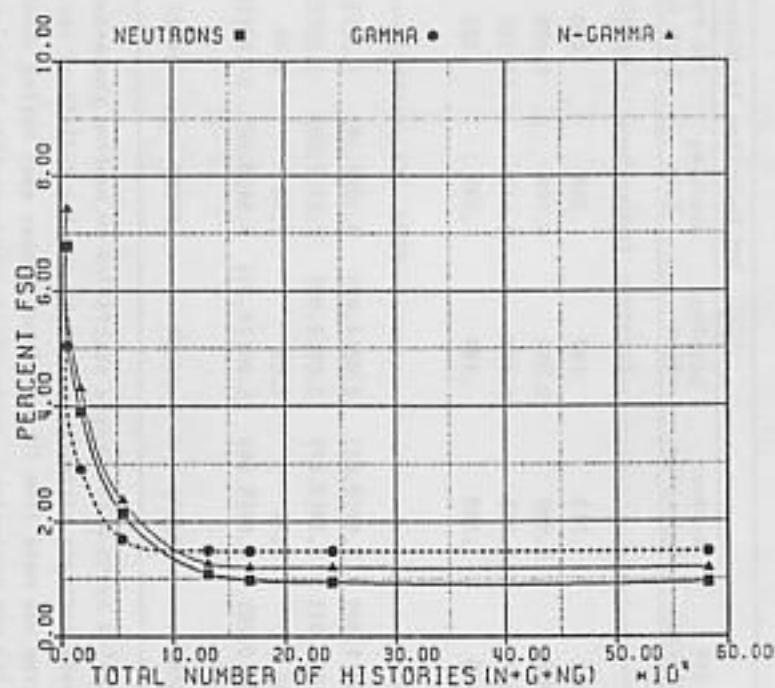


Figure 2. Percent fractional standard deviation as a function of history reduction. (N+G+NG)

The applicability of the optimization procedure at 1500 m to a different orientation, range, or city was investigated. Similar improvements were noted no matter where or how the phantom was coupled. The spectra or organ responses do not change in an amount significant enough to invalidate the optimization procedure.

Similar improvement in the dosimetry system occurs when the house leakage data sets are optimized. Many of the calculated leaking histories do not contribute much fluence in the house. Optimizing the house data to give a good spectrum at a reasonable cost in running

Table 15. Comparison of House Doses and FSDs with Various History Biasing Schemes, Hiroshima, 1500 m Ground Range. House Type 33. ( $\approx 77,000$  Leaking Histories)

	70% Reduction in Histories								June 84 System
	All Histories	Random Rejection	Leaking Bias	Starting Bias	10% FSD in Each Group	Leaking* Bias	Starting* Bias	5% FSD in Each Group	
-----Exposure (House Dose in Rad)-----									
Neutron	.271	.271	.269	.259	.248	.277	.271	.274	.261
Prompt Gamma	9.807	9.500	9.854	9.740	9.706	10.044	9.833	9.784	9.054
Delayed Gamma	12.177	11.599	12.029	12.177	11.755	12.773	12.275	12.155	14.085
House Gamma	.178	.180	.146	.193	.165	.177	.179	.178	.168
-----FSDs (In percent)-----									
Neutron	1.58(1.44)	2.69(2.63)	2.02(1.99)	6.33(6.26)	3.31(3.72)	2.31(1.99)	1.70(1.63)	1.72(1.65)	2.90
Prompt Gamma	2.19(2.01)	3.74(3.67)	3.09(2.90)	2.22(2.08)	3.20(2.83)	3.37(3.40)	2.73(2.33)	2.25(2.05)	4.09
Delayed Gamma	2.13	3.76	2.85	2.33	3.74	3.63	3.08	2.18	4.16
House Gamma	3.65(3.26)	6.62(5.95)	15.06(13.31)	7.58(8.28)	17.10(16.16)	3.80(3.40)	3.67(3.30)	3.65(3.26)	6.04
# of Histories	76750	23025	22000	14009	14927	21975	17179	44889	23000

\*Neutron Fluence multiplied by 50.0 to improve statistics on neutron components.

( ) values in parenthesis indicate estimated values by summing individual variances.

Note: House leakage tape was made from 10 individual houses each having about 25,000 histories.

Combining gives about 250,000 histories. Angle biasing reduces this to about 80,000 histories. Part of the FSD is due to the variation between individual houses  $\approx 1\%$ .



Table 16. Comparison of Organ Doses and FSDs with Various House History Biasing Schemes, Hiroshima, 1500 m Ground Range. House Type 33. Marrow ( $\approx 600,000$  Leaking Histories)

	70% Reduction in House Histories							
	All Histories	Random Rejection	Leaking Bias	Starting Bias	10% FSD in Each Group	Leaking* Bias	Starting* Bias	5% FSD in Each Group
-----Marrow Doses-----								
Neutron	.094(.093)	.092(.084)	.092(.087)	.077(.060)	.089(.081)	.092(.087)	.093(.092)	.094(.092)
Prompt Gamma	7.576(7.282)	7.877(7.551)	7.509(7.276)	7.602(7.271)	7.507(6.928)	7.707(7.309)	7.507(7.066)	7.590(7.285)
Delayed Gamma	8.819(8.910)	8.956(8.357)	8.736(8.669)	8.936(9.011)	9.117(8.416)	9.147(8.460)	8.999(8.647)	8.921(9.006)
House Gamma	.134(.140)	.135(.136)	.120(.106)	.141(.138)	.118(.109)	.132(.136)	.133(.140)	.134(.140)
Body Gamma	.115(.090)	.118(.086)	.116(.087)	.086(.030)	.117(.076)	.116(.087)	.107(.065)	.121(.087)
-----FSDs (in percent)-----								
Neutron	.97(4.05)	1.25(4.71)	1.06(3.75)	2.93(15.23)	1.43(6.47)	1.06(3.75)	1.28(4.43)	1.03(4.17)
Prompt Gamma	1.38(6.05)	1.64(7.06)	1.48(6.39)	1.38(5.97)	1.50(6.47)	1.61(6.89)	1.41(5.95)	1.37(5.98)
Delayed Gamma	1.48(5.61)	1.55(7.15)	1.46(5.72)	1.45(5.47)	1.68(7.90)	1.72(7.69)	1.52(5.71)	1.47(5.57)
House Gamma	1.39(4.73)	1.98(9.37)	1.67(16.92)	2.35(12.27)	4.80(21.60)	1.41(5.15)	1.40(4.79)	1.39(4.74)
Body Gamma	1.98(8.62)	2.18(9.41)	2.04(9.40)	14.33(35.62)	2.65(11.60)	2.04(9.40)	6.63(14.57)	2.19(10.80)

\*Neutron fluence multiplied by 50.0 to improve statistics on neutron components.

( ) values in parenthesis are results from the Dec 84 system marrow YCS leakage tape of  $\approx 30,000$  histories.

time was also investigated. Results showing the house exposure and FSD for a typical house leakage tape are presented in Table 15. The adjoint leakage data from these optimized houses were coupled with organ leakage tapes. Results are shown in Table 16. Perhaps the most important point to make from the results in Tables 15 and 16 is that the statistics of the house leakage data affect the statistics of the organ dose as it is derived from the in-house fluence. Furthermore, the most important house histories for the house exposure calculation may not be the most important histories for calculating the organ dose. It is probably not desirable to optimize the house tape for dose alone, since other parts of the house spectrum are responsible for body gamma-ray dose.

All of these biasing improvements require no modification in the dosimetry system. Thus, the changes made in the leakage tapes are transparent to the user of the system.

TABLE 15  
House Exposure and FSD for a Typical House Leakage Tape

House History	House Exposure (mSv)	FSD (mSv)
1	0.000	0.000
2	0.000	0.000
3	0.000	0.000
4	0.000	0.000
5	0.000	0.000
6	0.000	0.000
7	0.000	0.000
8	0.000	0.000
9	0.000	0.000
10	0.000	0.000
11	0.000	0.000
12	0.000	0.000
13	0.000	0.000
14	0.000	0.000
15	0.000	0.000
16	0.000	0.000
17	0.000	0.000
18	0.000	0.000
19	0.000	0.000
20	0.000	0.000
21	0.000	0.000
22	0.000	0.000
23	0.000	0.000
24	0.000	0.000
25	0.000	0.000
26	0.000	0.000
27	0.000	0.000
28	0.000	0.000
29	0.000	0.000
30	0.000	0.000
31	0.000	0.000
32	0.000	0.000
33	0.000	0.000
34	0.000	0.000
35	0.000	0.000
36	0.000	0.000
37	0.000	0.000
38	0.000	0.000
39	0.000	0.000
40	0.000	0.000
41	0.000	0.000
42	0.000	0.000
43	0.000	0.000
44	0.000	0.000
45	0.000	0.000
46	0.000	0.000
47	0.000	0.000
48	0.000	0.000
49	0.000	0.000
50	0.000	0.000